Disruption Modelling with M3D-C1

C. F. Clauser, S. C. Jardin, N. M. Ferraro

Princeton Plasma Physics Laboratory





• 2D – VDE studies in an ITER plasma: A Summary

Vertical Forces and the role of halo currents

- **3D VDE studies in an ITER plasma: progress status** Sideways forces (underway)
- Modelling C pellets in NSTX-U

Outline

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2D – VDE in ITER. A summary



APS Poster session: Wed 23rd Morning. PP10.00072

C. F. Clauser, S. C. Jardin and N. M. Ferraro, Nucl. Fusion 59 126037 (2019).

2D – VDE in ITER. A summary

We have scanned different post-TQ conditions by varying the post-TQ κ_{\perp}







C. F. Clauser, S. C. Jardin and N. M. Ferraro, Nucl. Fusion 59 126037 (2019).

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2D – VDE in ITER. A summary



- Larger halo current had larger $J_r B_{\phi}$ term, as expected,
- but, it is offset by a stronger reduction in the $J_{\phi}B_{r}$ contribution.
- Total vertical force is almost unaffected by magnitude of halo current.



The halo region formation produces a current density centroid displacement



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3D – **VDE Studies** (TQ also initiated by κ_{\perp})



Wall resistivity increased by 1000 $au_w \approx 0.235 \ {
m ms}$



3D – VDE Studies (TQ also initiated by κ_{\perp})

- Comparison of two simulations with $\tau_w = 235$ ms and $\tau_w = 0.235$ ms shows that small sideways force is due to $\gamma \tau_w >>1$ for n=1 mode with ITER vessel
- Large halo current case in progress
- Now in discussions with F. Villone about using CARRIDI to generate a more detailed wall model that can be incorporated into M3D-C1
 - Need to separate "first wall" and vessel
 - Need more detailed model of vessel structure



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Modelling C pellets in M3D-C1

- We are starting a systematic study of C pellet injection in a NSTX-Ulike configuration
- This is motivated by the EM pellet injector that is being proposed for NSTX-U
 - Very fast response time (2-3 ms)
 - Speeds up to 1 km/s

C ablation model based on...

- Sergeev et al., Plasma Phys. Rep. 32 (2006) 363
- Sergeev et al., ECA 18B (1994) 1364
- Kuteev et al., Sov. J. Plasma Phys. 10 (1984) 675

Neutral Gas Shielding Model (NGS)

- Key quantity is $\delta = q_p/q_0 \rightarrow$ shielding factor
- Hydrogen pellets
 - Low sublimation energy *ε*
 - $\delta \ll 1$: Most of the plasma heat flux is absorbed by the neutral pellet cloud
- Refractory pellets
 - High sublimation energy
 - $\delta \ge 0.8$: Most of the plasma heat flux reaches the pellet surface
 - Delayed time at which evaporation begins

The model does not include

- Suprathermal particle contribution
- Electrostatic shielding does not work well

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- Sergeev et al., Plasma Phys. Rep. 32 (2006) 363
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For both limits an analytical expression for the reaction rate \dot{N} is derived

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Strong shielding (\delta \rightarrow 0)
Based on scaling laws
\dot{N_0} \left[ \frac{\text{Atom}}{\text{s}} \right] \approx 1.94 \times 10^{14} n_e^{0.45} [\text{cm}^{-3}] \times T_e^{1.72} [\text{eV}] r_p^{1.44} [\text{cm}] \varepsilon^{-0.16} [\text{eV}] \times A_p^{-0.28} [\text{amu}] Z_p^{-0.56} (\gamma - 1)^{0.28}
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Weak shielding
$$(\delta \to 1)$$

 $\dot{N} \left[\frac{\text{Atom}}{\text{s}}\right] \cong \frac{\delta}{\varepsilon} r_p^2 n_e \sqrt{\frac{8\pi T_e^3}{m_e}}$
 $\delta^{-1} = 1 + \frac{1.725\sqrt{\pi}(\pi - 2)Z_p e^4 r_p n_e E_1 (I_{eff}/T_e)}{\varepsilon V_s \sqrt{2 m_e T_e}}$
 $V_s = \sqrt{\gamma T_s/m_p}$ $T_s \approx 5000 \text{ K}$

However, C pellet can have an intermediate shielding

- There is no analytical model for this regime
- They propose a standard interpolation:

$$\dot{N} = \frac{\dot{N}_0 \ \dot{N}_1}{\dot{N}_0 + \dot{N}_1}$$

Ablation models agree well with experimental data

They have been tested on a series of AUG shots

Parameter	3948	3950
r_p	0.25 mm	0.16 mm
v_p	485 m/s	265 m/s

Parameter	C atom	C ³ cluster
ε [eV]	8.8	11.6
γ	5/3	8/6
Z_p	6	18



SERGEEV et al.

Modelling C pellet in M3D-C1

We have incorporated these ablation models in M3D-C1

However, the spatial distribution for the neutral cloud is prescribed:

$$S = \frac{1}{(2\pi)^{3/2} V_p^2 V_t} \exp\left[-\frac{(R - R_p)^2 + (Z - Z_p)^2}{2V_p^2} - \frac{RR_p \left(1 - \cos(\varphi - \varphi_p)\right)}{V_t^2}\right]$$

In NSTX-U $R_{out} \sim 1.4$ m. Thus, the minimum toroidal neutral cloud size scales roughly as*

# tor. Planes	V _t
8	1.00 m
16	0.50 m
32	0.25 m



* Using spatially uniform toroidal planes. Now the code has the capability to increase the number of toroidal planes in a localized region. 15

Preliminary Studies

We have carried out preliminary simulations to evaluate how sensitive are the toroidal neutral cloud size

Parameter	Initial value
Pellet radius	2 mm
Pellet velocity	1000 m/s
Poloidal size V_p	10 cm



Next steps...

- We have requested the previous AUG shots with C pellets injection in order to Validate/calibrate our implementation.
 - This will be important to determine how large can be the neutral cloud

Assuming that ablated particles follow an adiabatic expansion: $T_s \approx 5000 \text{ K} \rightarrow V_s \approx 2.4 \times 10^5 \text{ cm/s}$ (Carbon atoms) $\nu_{ion} \gtrsim 10^6 \text{ s}^{-1}$ (C⁰ \rightarrow C¹ with kprad) Thus, the mean-free path $\lambda \leq 0.2 \text{ cm}$ This is a very small size for 3D modelling.

Next steps...

• Evaluate whether or not adding the initial pellet heating

• "The interaction of refractory pellet with plasmas differs substantially from that of Hydrogen pellets in that there is a noticeable delay between the time the pellet enters the plasma and the time at which evaporation begins" [Kuteev84]

Thus, for pellet with high enough sublimation energy – like C pellets – there is a non-zero time in which the pellet is heated before ablation.

This could be important for very fast pellet injection

$$C_P \frac{\mathrm{d}T}{\mathrm{d}t} = Q \longrightarrow T(t) = T_0 + \frac{1}{(4/3)\pi r_p^3 c_p} \int_0^t Q(t') \mathrm{d}t$$

Next steps...

- Carry out a series of simulations in NSTX-U scanning different parameters
 - Toroidal neutral cloud size
 - Using non-uniform toroidal plane distribution: increasing the spatial resolution around the pellet injection position
 - Different pellet velocities
 - Different ratio between the ablation to C and C³
- We are also interested in Be and W pellet injections into an ITER-like plasma



Extras...





Extras...







Because of the resistivity increasement at TQ, the plasma current starts decaying (**Current Quench**). This induces a total current in the wall. The CQ increases the force imbalance and the plasma speeds up its vertical drift.

0.6

time (s)

0.7

0.5

Z-magnetic Axis (m)

4.5

3.5

2.5

1.5

0.5

0.4

3

2

(C)



The currents flowing in the wall produce a total vertical force via **J**×**B**



Maximum vertical wall force for all the cases presented

