Ideal MHD limited electron temperature in (spherical) tokamaks

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From "soft beta limits" to "temperature flattening"

- The original motivation of this work was to better understand pressure-driven instabilities that do not cause disruptions in NSTX and other STs and tokamaks
- Examples of these are sawteeth and ELMs.
- But are there others, and if so: How do they saturate?
- Many NSTX discharges are predicted to be unstable to internal ideal MHD modes
- In performing many long-time nonlinear 3D MHD simulations, we saw that a common saturation mechanism for these internal ideal MHD modes is a flattening of the central electron temperature profile
- Could this "Ideal MHD" phenomena be responsible for the observed temperature flattening and associated thermal transport in NSTX?



Stutman, et al. PRL (2009)

 This large increase in the central χ_e with β has not been convincingly explained by micro-instabilities or energetic-particle driven transport

Theory Basis

"Infernal modes¹" are localized global internal ideal MHD instabilities that can occur in low shear regions at β -values well below the ballooning limit.

A recent paper by Boozer² shows that ideal MHD instabilities can lead to magnetic surface breakup, even for an arbitrarily small resistivity.

This opens up the possibility that surfaces can be destroyed in the vicinity of large pressure gradients, and that anomalous transport could occur by way of parallel diffusion in the resulting stochastic magnetic fields

We investigate this with the 3D MHD code $M3D-C^1$

¹Manickam, J., Pomphrey, N., Todd, A., "Ideal MHD stability properties of pressure driven modes in low shear tokamak" Nuclear Fusion (1987) ²Boozer, A., "The Rapid destruction of toroidal magnetic surfaces", Physics of Plasmas (2022)

3D Extended MHD Equations in M3D-C¹

$$\begin{split} \frac{\partial n}{\partial t} + \nabla \bullet (n\mathbf{V}) &= \nabla \bullet D_n \nabla n + S_n \\ \frac{\partial \mathbf{A}}{\partial t} &= -\mathbf{E} - \nabla \Phi, \quad \mathbf{B} = \nabla \times \mathbf{A}, \quad \mathbf{J} = \nabla \times \mathbf{B}, \quad \nabla_{\perp} \bullet \frac{1}{R^2} \nabla \Phi = -\nabla_{\perp} \bullet \frac{1}{R^2} \mathbf{E} \\ nM_i (\frac{\partial \mathbf{V}}{\partial t} + \mathbf{V} \bullet \nabla \mathbf{V}) + \nabla p &= \mathbf{J} \times \mathbf{B} - \nabla \bullet \mathbf{\Pi}_i + \mathbf{S}_m \\ \mathbf{E} + \mathbf{V} \times \mathbf{B} &= \eta \mathbf{J} + \mathbf{S}_{CD} \\ \frac{3}{2} \left[\frac{\partial p_e}{\partial t} + \nabla \bullet (p_e \mathbf{V}) \right] &= -p_e \nabla \bullet \mathbf{V} + \eta J^2 - \nabla \bullet \mathbf{q}_e + Q_\Delta + S_{eE} \\ \frac{3}{2} \left[\frac{\partial p_i}{\partial t} + \nabla \bullet (p_i \mathbf{V}) \right] &= -p_i \nabla \bullet \mathbf{V} - \mathbf{\Pi}_i : \nabla \mathbf{V} - \nabla \bullet \mathbf{q}_i - Q_\Delta + S_{iE} \\ \mathbf{\Pi}_i &= -\mu \left[\nabla \mathbf{V} + \nabla \mathbf{V}^\dagger \right] \qquad Q_\Delta = 3m_e (p_i - p_e) / (M_i \tau_e) \\ \mathbf{q}_{e,i} &= -\kappa_{e,i} \nabla T_{e,i} - \kappa_{\parallel e,i} \nabla_{\parallel} T_{e,i} \end{split}$$

Loop voltage at boundary, V_L , adjusted to keep I_P fixed. Sources: $S_n, S_m, S_{CD}, S_{eE}, S_{iE}$ Transport Coefs: $D_n, \mu, \eta, \kappa_e, \kappa_i, \kappa_{\parallel e}, \kappa_{\parallel i}$



These are the equations used in this study. Many other options available: Radiation, pellet ablation, conducting wall, reduced MHD, 2-fluid MHD, -K

Some of the results presented assumed $p_e = p_i$ for simplicity.

Code can be run in 2D or 3D

(2D should give very similar results as TRANSP with same sources and transport coefs.)

Unstructured Meshes used in this study



24 poloidal planes represented by Hermite cubic elements, error $\sim h^4$ Limited 48 plane runs done for convergence studies



- Triangular prism mesh is structured in toroidal direction, unstructured in poloidal plane.
- Use a high-resolution mesh so calculation can go to high S = 10⁸
- Variable size, highest resolution in center where S is largest
- Perform same calculation on 2 meshes for convergence study.
- High order C¹ finite elements error ~ h⁵ in (R,Z) plane

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Consider a typical reconstructed NSTX equilibrium



NSTX Shot 124379 @640 ms

 $\beta = 6.8\%$ $\beta_N = 3.9$ $I_P = 990$ kA $RB_T = 0.418$ T-m q(0) = 1.29



Central temperature : Te = 916 eV

Spitzer resistivity gives $S = 5 \times 10^7$ (at center_

geqdsk equilibrium linearly unstable to many ideal modes



We have run this case non-linearly up to $6000 \tau_A$ with no sources and small transport coefficients

Poincare Plots \rightarrow

Nonlinear Development of surfaces and temperature



Change in Temperature from time t=0 \rightarrow

(t = 6000 corresponds to 2.75 ms)

G46F

Summary of 0 < t < 1200 τ_A





Unstable (4,3) mode grows up, breaks magnetic surfaces near and interior to rational surface, causing central temperature and pressure to decrease, stabilizing plasma

$$D_n, \mu, \kappa_e, \kappa_i = 10^{-6}$$
 $\kappa_{\parallel e}, \kappa_{\parallel i} = 10$ $\eta =$ Spitzer 12

With sheared rotation (25 kH in center) results are qualitatively similar



Including sheared rotation smooths final temperature profile (shown at t=3200 τ_A or 1.90 ms)

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Generate a family of 3 equilibrium by Bateman scaling

A Bateman scaling factor of 0.9 (10% weaker toroidal field) produces a more unstable equilibrium with q(0) = 1.2 and $\beta = 8.2$ %

BS = 1.1 (stronger TF) is almost stable to all modes q(0)= 1.4, β = 5.8 %

Shown on right are the linear stability properties of the 3 equilibrium



Next, evolve the (more unstable) BS=0.9 equilibrium nonlinearly

Bateman scaling keeps the current density fixed (P' and FF') but varies the toroidal field to generate a family of equilibrium from a given geqdsk file

Bateman scaled equilibrium with BS=0.9, q(0) = 1.2, $\beta = 8.2\%$ also saturates but final state has multiple n modes (5,4) most unstable Magnetic Energy in Toroidal Harmonics 0.00035 0.00030 n=1 0.00025 n=2 1000 14 n=3 า=4 0.00020 Initial n=5 12 Final n=6 Initial 0.00015 800 Temperature (eV) Final 10 Safety Factor (q) 0.00010 600 8 0.00005 0.00000 6 400 5000 6000 3000 0 1000 2000 4000 Electron Time τ_{Δ} Magnetic Energy in Toroidal Harmonics 5e-6 200 2 4e-6 0 0 0.6 1.6 0.0 0.2 0.8 0.4 0.8 1.0 1.2 1.4 0.4 0.6 n=3 3e-6 Major Radius Normalized Ψ n=5 n=6 2e-6 Note: n =3,4,5,6 toroidal harmonics all comparable at end 1e-6 q-profiles does not change during evolution Axis temperature greatly reduced during evolution 0 1000 2000 3000 4000 5000 6000 16

G46-F3

Time τ_{Δ}

Comparison of the time evolution of the 3 scaled equilibria



G46F4 - G46F - G46F3

Comparison of 3 Stutman shots and 3 BS equilibria



- Top is the 3 shots analyzed in the Stutman PRL
- Bottom are the 3 Bateman-scaled equilibria nonlinearly evolved with M3D-C1
- These were not the same shots, but the trends are similar
- Te most peaked at low $\beta_{\text{N}}.$ Increasing β_{N} results in broader profiles

Stutman, et al. PRL (2009)

M3D- C^1 shows similar scaling with B_T as experiment



Note: Plot on left is Te(0). On the right is τ_{E}

G46F4 - G46F - G46F3

Kaye, et al, PRL (2007)



"Some of the discharges in this study did exhibit both low amplitude low-n MHD activity as well as the fast ion driven Alfven eigenmode (AE) activity,"¹⁹

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More realistic: Start with stable equilibrium and apply heating power: First in 2D



Results (in 3D) will be shown in 2 phases In first phase, 0< t < 3000 $\tau_{A,}$, β increases to 7%

In first phase, only minor temperature transport near q=1.5





Summary of Temperature Profiles



After t=4000, 3D central temperature no longer increases, but temperature profile broadens.

In 2D, central temperature continues to rise.

Plot of Central Te vs β during heating sequence – 2D vs 3D



Central temperatures in 2D and 3D are the same for β < 8%. For β > 8%, Te(0) does not increase with β

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Shot <u>121014@0.51</u> s (From S. Kaye series of $\tau_{\rm E}$ scaling with B_T)



Near stationary state sequence with rotation





- BS=1 case with low heating power and torque drive
- κ_{\perp} = 1.e-5, $\kappa_{||}$ = 10 (Te only)
- Strengths of sources chosen to make 3D case approximately stationary
- Comparison of 2D and 3D case with same transport coefficients shows affects of 3D instability
- In 3D, β is slightly lower and Te(0) is significantly lower
- Te more affected than Ti

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Does this occur in high- β discharges in conventional aspect ratio tokamaks?

- DIII-D, Turnbull, et al. Fusion Science and Tech. 48 (2005)
 - Infernal modes unambiguously identified in high β_P discharges, inversely correlated with periods of improved confinement
- JET, Charlton, et al, Nucl Fusion **31** 1835 (1991)
 - Pellet fuelled shots with peaked pressure profiles terminated by an abrupt flattening of the temperature profile. (3,2) infernal mode when q(0) drops below 1.5
- TFTR, Chang, et al, Nucl. Fusion **34** 1209 (1995)
 - Supershot performance degradation in presence of (3,2) and (4,3) macroscopic modes
- JT-60, Ozeki, Nucl. Fusion 35 861 (1995)
 - In high *I*_i plasmas with peaked pressure, the stability limit is determined by infernal modes in the low q₀ regime

Summary

- NSTX equilibrium 124379 @640 ms found to be linearly unstable to ideal MHD modes that saturate at modest amplitude with small non-axisymmetric n=3 toroidal harmonic that flatten Te
- Higher beta equilibrium obtained by Bateman scaling are much more unstable linearly, and saturate with n=3,4,5,6 toroidal harmonics which lead to stochastic surfaces and more Te flattening
- Calculations performed with actual Spitzer resistivity, $S = 5 \times 10^7$ and with variable fine meshes
- More realistic calculations start with lower-beta (stable) equilibria, apply heating source to drive it through the beta limit. Stays at marginal stability in center by broadening the temperature profiles and distorting the flux surfaces..
- Future:
 - 1. We are examining other high- β NSTX equilibria to see how universal this effect is? Are most high β equilibrium in a nonlinearly saturated state? Implications for transport analysis.
 - 2. Longer term: Role of energetic particles with $M3D-C^{1}-K$
 - 3. What does this say about preferred operational regimes in NSTX-U? Can we operate the machine in such a way as to minimize the deleterious effects of these modes?

Extra VGs

Effect of Sheared Rotation ~ 20 kHz in Center



Trend is similar to experiments on NSTX



- M3D-C1: Central temperature decreases with β ullet
- Exp data: Central transport increases with β ullet

G46F4 - G46F - G46F3

133964: LRDFIT and TRANSP files have different profiles and stability properties

